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A high-repetition rate edge localised mode replication system for the Magnum-PSI and Pilot-PSI linear devices

T W Morgan¹, T M de Kruif^{1,2}, H J van der Meiden¹, M A van den Berg¹, J Scholten¹, W Melissen¹, B J M Krijger¹, S Bardin¹ and G De Temmerman¹

E-mail: t.w.morgan@differ.nl

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Abstract

A high-power edge-localized mode (ELM) striking onto divertor components presents one of the strongest lifetime and performance challenges for plasma facing components in future fusion reactors. A high-repetition-rate ELM replication system has been constructed and was commissioned at the Magnum-PSI linear device to investigate the synergy between steady state plasma exposure and the large increase in heat and particle flux to the plasma facing surface during repeated ELM transients in conditions aiming to mimic as closely as possible those in the ITER divertor. This system is capable of increasing the electron density and temperature from $\sim 1 \times 10^{20} \,\mathrm{m}^{-3}$ to $\sim 1 \times 10^{21} \,\mathrm{m}^{-3}$ and from 1 to 5 eV respectively, leading to a heat flux increase at the surface to $\sim 130 \, \mathrm{MW \, m^{-2}}$. By combining Thomson scattering measurements with heat fluxes determined using the THEODOR code, the sheath heat transmission factor during the pulses was determined to be \approx 7.7, in agreement with sheath theory. The heat flux is found to be linearly dependent upon the strength of the magnetic field at the target position, and, by adapting the system to Pilot-PSI, tests at 1.6 T showed heat fluxes of more than 600 MW m⁻². This gives confidence that with the installation of a 2.5 T superconducting magnetic solenoid at Magnum-PSI the heat flux will reach the ITER-relevant gigawatt per square metre heat flux regime.

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Keywords: ELMs, ITER, Magnum-PSI, Pilot-PSI

(Some figures may appear in colour only in the online journal)

1. Introduction

The lifetime and heat removal capability of the ITER divertor and those of future high-performance fusion devices remains one of the most crucial questions to address on the path to fusion power. During normal operation the ITER divertor is expected to receive a steady heat and particle flux of $\sim 10 \, \mathrm{MW \ m^{-2}}$ and $\sim 10^{24} \, \mathrm{m^{-2}}$ s⁻¹ [1–3]. Under such fluxes strong modification of the plasma-exposed surface can occur, such as nano-structure formation under helium bombardment [4, 5] and blistering under hydrogen and deuterium plasma exposure [6, 7]. In combination with this, edge-localized

modes (ELMs) will add short periodic bursts of heat and particles. For type-I ELMs power densities of up to 1–10 GW m⁻² with a duration between 0.5 and 1 ms and frequency 1–10 Hz are expected in ITER [8]. These conditions are not replicated in current tokamak devices, so their study requires specialized laboratory experiments. Combined experiments with steady state linear plasmas and pulsed high-power laser irradiations at NAGDIS-II [9] and PISCES-A [10] show a reduced damage threshold for tungsten, compared to ELM-replication experiments alone. This suggests a synergistic effect between the steady plasma flux and the transient loading of the surface. This is also supported by

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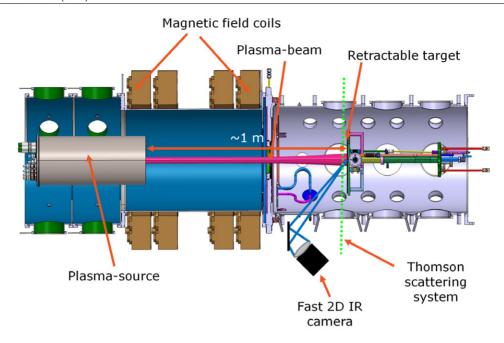


Figure 1. Schematic diagram of the Magnum-PSI device showing the positioning of the TS and fast-framing infra-red camera.

experiments in Pilot-PSI using a capacitor bank discharged through the plasma source to produce a transient pulse of not only high heat flux but also high particle fluxes, which well replicated the combined steady and transient conditions expected in the ITER divertor [11, 12]. The success of these experiments motivated the development of an improved system which could be used for both the Magnum-PSI and Pilot-PSI linear devices, designed as a multi-pulse ELM replication system (MERS). The main innovations are a high repetition rate (>100 Hz), compared to the previous single-pulse per discharge system, and flexible pulse shape through varying the pulse timing and voltage of different independently controlled capacitors. The development and characterization of this system as well as initial results will be presented in this work.

2. Experimental set-up

Due to its superior diagnostic capabilities and flexibility the Magnum-PSI device was chosen for the commissioning and characterization of the MERS system. Magnum-PSI utilizes a cascaded arc discharge [13, 14] confined into a narrow beam (FWHM ~25 mm) by magnetic coils producing up to 1.3 T, which impacts the target ~1 m from the source, as described in detail in [15] (figure 1). At the target position the machine produces a low-temperature ($T_e \sim 1$ –3.5 eV), high-density ($n_e \sim 1$ –7 × 10^{20} m⁻³) plasma with a particle and heat flux of $\Gamma \sim 1 \times 10^{24}$ m⁻² s⁻¹ and ≤ 12 MW m⁻² respectively [16, 17]: conditions which well replicate those expected in the ITER divertor. The surface temperature was monitored using a fast-framing infra-red camera (FLIR SC7500MB) used at ~5 kHz, while the plasma density and electron temperature were determined via Thomson scattering (TS) (section 4).

The cascaded arc source operates with a current typically between 100 and 200 A from a dc current-regulated source. In parallel with this, a modular capacitor bank system

This consists of 28 identical sections as was installed. shown in figure 2. Each section consists of an inductor (160 μ H), a capacitor (150 μ F), externally triggered charging and discharging switches, and an anti-reversal diode. After the capacitor has been charged to the required voltage the stored energy can be dissipated through the plasma's resistance, leading to both Ohmic heating and increased ionization, and transiently producing higher temperatures and densities. Each capacitor can be charged up to 3000 V, with a total maximum stored energy of \sim 19 kJ for the entire system. Each section can be charged and discharged independently with a charging time of 22.2 ms, permitting high repetition rates above 100 Hz to easily be achieved by discharging in sequence. Further, by varying the delay time between the discharges of several capacitors, the rise time can be changed and the pulse shaped. Alternatively, the total energy of the pulse can be increased by discharging several capacitor banks simultaneously. Therefore, a combination of steady state plasma and transient pulses of increased heat and particle flux is created, analogous to the situation expected in the ITER divertor.

The cascaded arc source used was designed to accommodate the high power densities of the transient pulses by using a wide channel diameter (10 mm) lined with molybdenum inserts to reduce impurity production as described in [12]. Due to the increased average heat flux to the nozzle due to repeated pulsing, further protection of the surface was required to protect it from melting and impurity production. Therefore, the nozzle plate was changed from Cu to a CuW alloy (25:75) coated with a 200 nm thick W layer. This thickness was chosen to avoid flaking, while still maintaining a good coating lifetime.

The current pulse through the source can be described in two modes—the resonance mode and the decay mode. In the resonance mode the anti-reversal diode is non-conducting and the discharge through the plasma forms an *RLC* circuit. As the

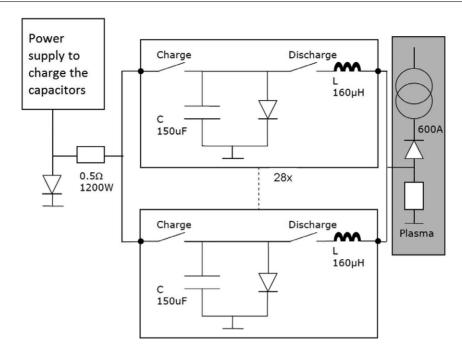


Figure 2. Schematic picture of the capacitor bank system used in Magnum-PSI.

circuit is highly underdamped (ζ < 0.1), the resulting current curve can be described by the first quarter of a sine wave,

$$I(t) = I_0 \sin\left(\frac{t}{\sqrt{LC}}\right)$$
 for $0 < t < \frac{\pi}{2}\sqrt{LC}$, (1)

where I_0 is the peak current of the system and L and C the total inductance and capacitance of the system, respectively.

The decay mode occurs when the current is at its maximum and the voltage over the capacitors is zero. At this point, the anti-reversal diode becomes conducting and the section functions as an LR circuit. The resulting current through the plasma source then becomes

$$I(t) = I_0 e^{-\frac{Rt}{L}} \qquad \text{for } t > \frac{\pi}{2} \sqrt{LC}, \tag{2}$$

where *R* is the total resistance of the system.

3. Performance of the pulsed system

The current and voltage characteristics of the pulses were monitored using a Rogowski coil (Hioki 9667) and voltage probe (LeCroy PPE) respectively. A typical discharge is shown in figure 3. In this case the current rise time for the pulse is $\sim\!220\,\mu s$, while the e-folding time for the current decay is $\sim\!600\,\mu s$. From these $I\!-\!V$ traces the dissipated power in the source can be established, as shown in figure 4. This demonstrates two trends—first that the peak power increases non-linearly with stored energy, while the characteristic decay time ($\tau_{\rm decay} = L_{\rm total}/R_{\rm total}$) increases with stored energy, following a similar trend. In all cases the rise time is approximately constant at around $210\,\pm\,10\,\mu s$. This is summarized in figure 5.

By integrating the power evolution over the time of the discharge, the energy dissipated in the plasma can be

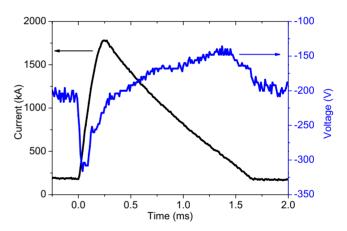


Figure 3. Typical current (dashed) and voltage (solid) trace with a capacitor stored energy of 320 J. The potential was measured between the cathode and the grounded anode of the plasma source, while the current was measured from the capacitor bank with the dc current (190 A) incorporated.

determined and the efficiency of the pulsed source calculated as $\eta = \frac{E_{\text{Dissipated}}}{E_{\text{Stored}}} = 92.1 \pm 0.8\%$ (figure 6) for a single section. This demonstrates that efficiency is constant for all stored energies, so the increasing decay time balances the non-linear increase in peak power. This increase of decay time can be explained through the decreasing resistance of the plasma as a function of input energy. As most of the power is dissipated in the source through Ohmic heating, the plasma resistance decreases with increasing electron temperature as [18]

$$\eta_{\rm e} = 2 \times 10^4 (kT_{\rm e})^{-3/2} \ln \Lambda$$
 (3)

for a highly ionized hydrogen plasma. The increasing decay time can be fitted as $\tau_{\rm decay} \propto R_{\rm min}^{-1.02\pm0.02}$, demonstrating that this increase is entirely due to a changing plasma resistivity.

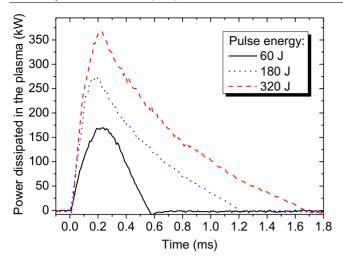


Figure 4. Power evolution dissipated in the plasma as determined from *I* and *V* measurements for different stored energies.

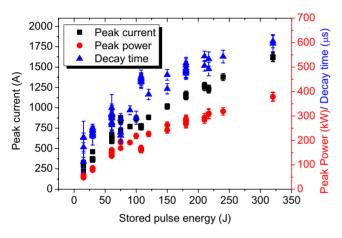


Figure 5. Peak current, peak power and decay time as a function of the stored energy of the pulse, showing that all three evolve in a similar non-linear way.

The decay time is defined by

$$\tau_{\text{decay}} = \frac{L_{\text{section}}/N + L_{\text{plasma}}}{R_{\text{plasma}} + R_{\text{circuit}}},$$
 (4)

where N is the number of sections, so by adding further capacitor sections the pulse can be shaped. Setting $L_{\rm plasma}$ and $R_{\rm circuit}$ to zero gives reasonable agreement, demonstrating that the capacitor inductance is the dominant factor (figure 8). Including them gives values for the plasma inductance of $43.6 \pm 1.6 \, \mu \rm H$ and the circuit resistance of $0.063 \pm 0.022 \, \Omega$. While these values are significantly lower and thus less significant than the values of the section inductors and the plasma resistance respectively, they still play a small role in determining the circuit response to the discharge.

Examination of the peak power deposited as a function of section number, however, shows that, while initially this leads to a higher peak power, this value rapidly decreases as section number is increased. This is due to the non-perfect efficiency of each section, which leads to a reduced efficiency of the total system as $\eta_{\text{system}} = \eta_{\text{section}}^N$ (figure 8). Fitting this to the observed data gives $\eta_{\text{section}} = 92 \pm 14\%$, which is close

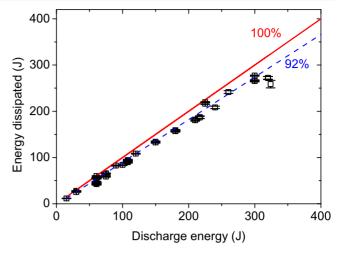


Figure 6. Dissipated energy in the source as a function of capacitor stored energy showing fitted efficiency of 92.1% and the theoretical maximum of 100% efficiency.

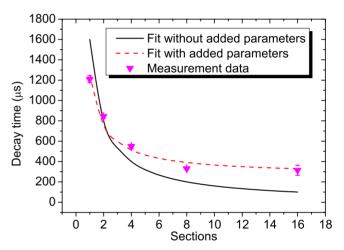


Figure 7. Decay time as a function of capacitor bank section compared against calculated values using the measured minimum resistance for a constant stored energy of 480 J. Also shown are fits to the decay time using either $\tau_{\text{decay}} = L_{\text{section}}/NR_{\text{total}}$ (solid) or equation (4) (dashed). Data were recorded during operation at Pilot-PSI (see section 6).

to the previously measured values. Therefore, it is necessary to increase the stored energy in each capacitor to compensate for this effect.

4. Evolution of the plasma parameters during a pulse

The electron density (n_e) and temperature (T_e) of the plasma as a function of beam radius can be determined using a TS system in Magnum-PSI located at a distance $\geqslant 15$ mm from the target position [19]. The integration time of the TS detection system is much shorter (~ 25 ns) than the pulse duration, so n_e and T_e can be determined during a particular moment of a pulse within a single integration time. The high electron density during a pulse gives a large signal and thus enables this. By simultaneous triggering of both systems and by utilizing a variable delay it is possible to measure a different time interval in the pulse relative to its start. Due to the

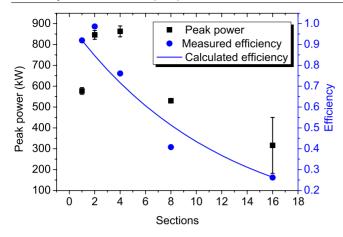


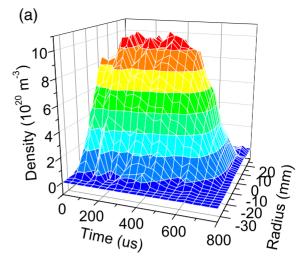
Figure 8. Peak power as a function of number of capacitor sections used for a fixed stored capacitor energy of 480 J. Initially increasing capacitor number leads to a reduced decay time and higher peak power, but as the number of capacitors is increased the efficiency of energy transfer to the plasma is reduced, leading to a lower overall peak power. Comparison of the calculated efficiency with the measured values shows reasonable agreement. Data were recorded during operation at Pilot-PSI (see section 6).

high replicability of each pulse, this can be used to build up a temporal and spatial picture of the pulse through the shot by systematically increasing the delay each time. Typical density and temperature evolutions are shown in figure 9. All measurements were made with an H_2 input gas flow rate of $17 \, \text{Pa} \, \text{m}^3 \, \, \text{s}^{-1}$, a dc current of $200 \, \text{A}$ and a magnetic field in the coil centre of $1.3 \, \text{T}$.

During the increased power input to the plasma during the pulse, two processes are important. First, the temperature of the plasma increases due to Ohmic heating of the electrons, such that the temperature increase closely follows the power trace. Due to this temperature increase the ionization fraction increases dramatically and the electron density quickly reaches a saturated value, consistent with previous observations of increased electron density with gas flow in Pilot-PSI [20]. An estimation based on the neutral gas flow rate (17 Pa m³ s⁻¹) and the Saha equation for hydrogen gives an ionization fraction above 99% for $T_e > \sim 1.8$ eV. This agrees well with the high and saturated values for n_e observed at temperatures above this. Similarly, a precipitous decline in electron density is observed as the temperature falls to low values at the end of the pulse.

In figure 10(a), the evolution of the electron temperature at the beam centre (r=0) as a function of time is shown for different stored energies (all pulses used a single section). The pulse length and peak temperature increase in the same manner as the power trace. The temperature at the time of maximum current is plotted in figure 10(b). For low pulse currents (<500 A) the temperature increases strongly compared to the dc temperature, but for higher currents this increase becomes incrementally smaller. This is due to the decreasing effect of the Ohmic heating as the resistance decreases, while the electron heat conduction losses increase strongly as a function of the peak electron temperature ($Q_{\rm e}^{\rm cond} \propto T_{\rm e}^{7/2}$).

By equating the Ohmic heating $(\propto j^2/T_{\rm e}^{3/2})$ to electron heat conduction losses as in [12], the electron temperature is shown to scale as $T_{\rm e} \propto j^{2/5}$. Allometrically fitting this value



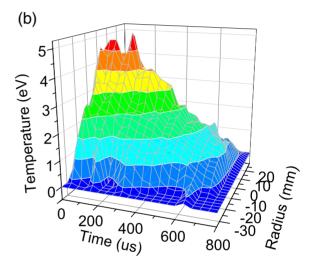


Figure 9. 3D plots of the temporal and spatial evolution of (a) electron density and (b) electron temperature during a typical 60 J stored energy plasma pulse, as measured by TS 18 mm in front of the target. These were created using 32 plasma pulses discharged during a single shot with the TS performed at a time interval $25 \,\mu s$ earlier each time relative to each pulse.

to the data shows excellent agreement, indicating that these processes are the dominant ones during the pulse.

Similarly to figure 10, the evolution of the electron density at the beam centre as a function of time is shown for the same set of pulses in figure 11(a). This demonstrates that the density is saturated at approximately 1×10^{21} m⁻³ and that the duration of the saturated density matches well with the time duration of the pulse and with the temperature evolution. As the deposited energy increases, the peak density reaches a lower saturation value. As the total flux is limited by the gas flow input into the source, the particle flux $\Gamma = n_{\rm e}v_{\rm th}$ can be assumed to be a conserved quantity in the case of full ionization. As the temperature increases, the electrons and ions pass the observation position at an increasing average speed and with a larger range of Maxwellian energies, producing a relative rarefaction of the plasma pulse. The maximum probability distribution for the speed decreases as a function of temperature as $T_{\rm e}^{-1/2}$. Therefore, the maximum electron density should decrease with current as $j^{-1/5}$. Figure 11 shows the peak value

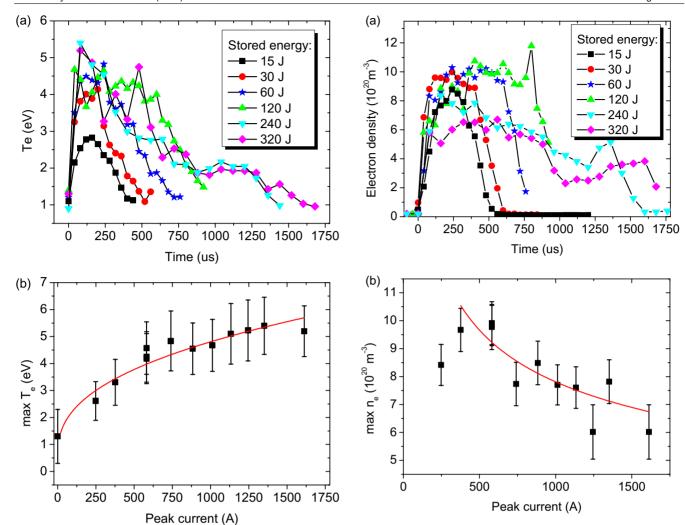


Figure 10. (a) Evolution of electron temperature at r=0 during the pulse for different stored energies utilizing a single capacitor bank section. (b) Value of the electron temperature at the time of maximum current (typically $210 \, \mu s$ after the start of the pulse) as a function of maximum current. The fitted line is of the form $T_{\rm e}=Aj^{2/5}$, where A is a free fitting parameter.

of $n_{\rm e}$ at the time of maximum current. The data where density saturation has been reached ($E_{\rm stored} > 15\,\rm J$) are allometrically fitted and show a power dependence of -0.307 ± 0.062 , which therefore shows reasonable agreement within the fitting error.

5. Evolution of the target parameters during a pulse

The increased plasma density and temperature in turn increase the surface temperature of the target transiently each time. A temperature trace as recorded using the FLIR infra-red camera is shown in figure 12(a) for a typical discharge. Here the magnetic field is turned on at t=0 s, leading to a surface temperature increase to $\sim 480\,^{\circ}$ C. The slight decline in baseline temperature through the discharge after the initial heating phase originates from a slow decrease of the magnetic field strength of the coils due to their heating (and also limits the duration of the discharge). Each firing of the capacitor leads to an average temperature increase of $173\pm 8\,^{\circ}$ C. Heat fluxes can be determined from the temperature evolution using the

Figure 11. (a) Evolution of electron density at r=0 during the pulse for different stored energies utilizing a single capacitor bank section. (b) Value of the electron density at the time of maximum current (typically $210\,\mu s$ after the start of the pulse) as a function of maximum current.

numerical heat flux solver THEODOR [21]. This solves the heat flux equation in two dimensions for a flat surface

$$\rho C \frac{\partial T(x,t)}{\partial t} = \nabla (k \nabla T(x,t))$$
 (5)

from the temperature distribution T(x,t) across the surface x, with ρ , C and k the density, heat capacity and thermal conductivity of the bulk material respectively. A more thorough description of the code is given in [22]. Heat transfer coefficients on the top (α_{top}) and bottom (α_{bottom}) of the sample act as boundary conditions to be optimized for and are given by

$$q_{\text{plasma}} = \alpha_{\text{top}} \left(T_{\text{IR}} - T_{\text{bulk}} \right) \tag{6}$$

and

$$q_{\text{cooling}} = \alpha_{\text{bottom}} \left(T_{\text{bulk}} - T_{\text{cooling}} \right).$$
 (7)

 α_{bottom} was determined by iteratively minimizing the heat flux after the discharge to zero (and has little effect upon the transient heat flux determination), while α_{top} was found using the methodology described in [23], leading to $\alpha_{bottom}=80\,kW\,m^{-2}\,K^{-1}$ and $\alpha_{top}=0.8\,MW\,m^{-2}\,K^{-1}$. The

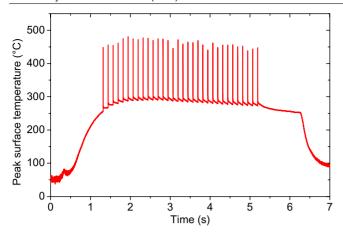


Figure 12. Temperature evolution of a tungsten target at the position r = 0 subjected to a 200 A, 17 Pa m³ s⁻¹ plasma with a 0.9 T field in the bore of the magnet. 8 Hz pulses with 55 J deposited energy were superimposed by capacitor bank discharge.

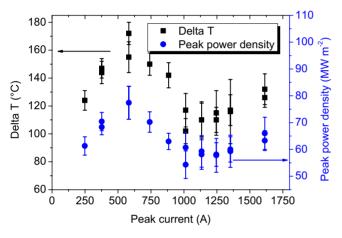


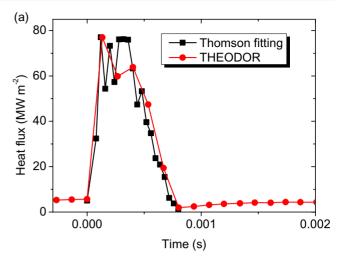
Figure 13. Peak temperature increase (squares) and maximum heat flux to the surface as determined from the THEODOR code (circles) due to a transient pulse as a function of peak input current to the source. Pulses range from 15 to 320 J stored energy.

value for α_{top} represents the effects of surface roughness producing a thin layer of reduced conductivity at the surface, and is required to avoid negative heat fluxes after the transients [24–26]. Analysis of the same discharges with the ANSYS package using finite element analysis of the full 3D geometry of the target [27] demonstrates that a thin barrier layer is indeed required to avoid this effect.

The temperature increase of a tungsten sample and the plasma heat flux at r=0 as a function of peak current are shown in figure 13. As the ramp-time of the pulse is approximately constant in all cases, the relationship between temperature and heat flux is a linear one and therefore both follow the same form as the source current increases, as $P\sqrt{t_{\rm rise}} \propto \Delta T$ [28]. The heat flux to the surface can also be defined as

$$q = \gamma k_{\rm B} T_{\rm e} \Gamma_{\rm se} \approx \left(\frac{2}{3m_{\rm i}}\right)^{1/2} \gamma n_{\rm e} (k_{\rm B} T_{\rm e})^{3/2},$$
 (8)

where γ is the sheath heat transmission coefficient (incorporating contributions from ions), Γ_{se} is the ion flux to the surface and m_i the ion mass, and the specific heat ratio is



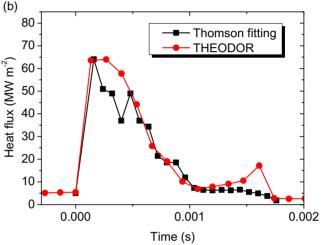


Figure 14. Evolution of the heat flux at r=0 calculated either from equation (8) with $\gamma=7.7$ (squares) or from the THEODOR code (circles) for pulses with either (a) 60 J stored energy or (b) 320 J stored energy.

assumed to be 5/3 [29]. From this, figure 13 can be understood from the evolution of electron density and temperature as a function of current. The initial strong increase in heat flux for currents below 600 A results from the combined increase in both $n_{\rm e}$ and $T_{\rm e}$. Between 600 and 1250 A the heat flux values fall, as the values of $n_{\rm e}$ fall sharply in this range due to the saturation in ionization and pulse rarefaction. For values above 1250 A the temperature increase (scaling in equation (8) as proportional to $\sim I^{3/5}$) overcomes the decrease in $n_{\rm e}$ (scaling as proportional to $\sim I^{-1/5}$). Therefore the heat flux should scale with $\sim I^{2/5}$ for large currents. Allometric scaling of the form $q = AI^b$ gives the power dependence with respect to current as $b = 0.42 \pm 0.04$ in the range 1250–1600 kA, which is in good agreement with this.

As time-resolved TS measurements are available for these measurements, these provide a way to determine the value of the sheath heat transmission factor γ . In figure 14 measured heat flux is compared with the expected values of heat flux to the surface (q) as calculated from equation (8) for two example pulse conditions. The values of $n_{\rm e}$ and $T_{\rm e}$ as determined in section 4 were used while keeping γ as a free parameter. This calculation shows good agreement in the shape of the pulse

in comparison with the THEODOR calculation and gives a value $\gamma \approx 7.7$ for all compared pulses. From sheath theory, γ can be calculated as in [30] (where we assume $T_{\rm i} \approx T_{\rm e}$ based on Doppler broadening of Balmer β (486.1 nm) emission [12,31]). For the electron temperature range of 2.8–5.9 eV (the range of measured values), the values of γ decreases from 9.3 to 7.3, indicating that these results are consistent with expectations.

6. Discussion

The measured heat fluxes using MERS are significantly lower than those measured in Pilot-PSI using the original capacitor bank system where heat fluxes up to 1.2 GW m⁻² were achieved [11]. These values were achieved with a source current of ~12 kA however, while the maximum investigated here was 1.7 kA to ensure the good lifetime of the plasma source (as the MERS system aims at investigating the effect of large numbers of ELM-like pulses on material surfaces). A second reason for the lower heat flux is that the magnetic field of Magnum-PSI diverges beyond the position of the solenoids, while the target is located 0.6 m from the exit of the coils. Therefore, at the target position the maximum value of the magnetic field (B_t) is 0.3 T rather than 1.3 T within the coil region (B_c) . Due to this expansion, from the frozen-in condition and Gauss' law, the beam size is likewise expanded, i.e. $B_c A_c = B_t A_t$. From particle conservation the particle flux density is thus reduced, leading to a reduction in heat flux to the target (equation (8)). We can describe this as

$$\frac{q_{\rm t}}{B_{\rm t}} = \frac{q_{\rm c}}{B_{\rm c}}.\tag{9}$$

To investigate this effect further the Magnum-PSI magnetic field was varied from 0.15 to 0.3 T at the target position (by varying the coil current) utilizing a capacitor stored energy of 320 J, leading to a peak source current of 1.6 kA. This led to a doubling of the heat flux to the surface, as expected from equation (9) (figure 15). These values can be compared with those measured in Pilot-PSI with the single-pulse system [32], as the source current used was similar (1.7 kA) and plasma conditions were identical in gas flow and dc current supply. This shows that the heat flux values are comparable, while being approximately twice as high as those that can be extrapolated from the Pilot data for a similar magnetic field strength. However, in [32] the source channel diameter was smaller, which may account for the difference.

To determine any differences in the performance of MERS and the single-pulse capacitor bank system, MERS was coupled to Pilot-PSI using an identical source. The pulse currents and voltages were compared to those measured in Magnum-PSI and found to be extremely similar, indicating that within the source the performance of the system is identical in the two machines. As a more complete data-set was available from Pilot experiments, the I-V characteristics recorded were used in section 3 where indicated. Heat flux measurements in Pilot-PSI using MERS with a source current of 1.4 kA and a magnetic field of 1.6 T are also shown in figure 15, and show good agreement with the earlier Pilot-PSI data.

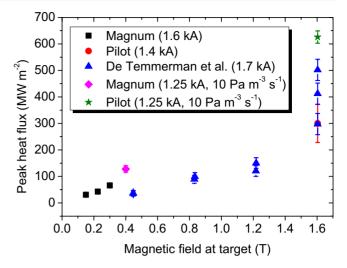


Figure 15. Peak heat flux measured at r=0 as a function of magnetic field at the target position either in Magnum-PSI using MERS by changing the overall magnetic field strength (squares) or by moving the target towards the source (diamonds), or in Pilot-PSI using the single-pulse system (triangles) or MERS (circles and stars). The single-pulse results are from De Temmerman *et al* (2010) [32].

Reducing the gas flow in the source has the effect of reducing the electron density and increasing the electron temperature of the plasma exiting the source [33], due to the lower number of hydrogen atoms to be ionized and thus the higher energy per atom available to heat it. As the heat flux density is a stronger function of temperature than density, an inversely proportional change can elicit a stronger heating effect. Reducing the gas flow rate also reduces the background pressure in the vessel, which in turn reduces molecular assisted recombination (MAR), which is strongly pressure dependent [34]. As MAR is a most effective recombination mechanism for electron temperatures between 0.5 and 2 eV [35], for the pulse MAR plays a role only in the edges of the beam, but some reduction in recombination is expected. To attempt to utilize these two effects the input gas flow in Pilot was reduced from 17 to 10 Pa $\rm m^3~s^{-1}$, and thus the background pressure from 10 to 6 Pa $\rm m^3~s^{-1}$. In this case, the heat flux deposited was increased to 626 ± 23 MW m⁻², even in the non-optimized case of 1.25 kA peak current. As time-resolved TS as described in section 4 was not available, it was not however possible to distinguish the contributions from changing temperatures and electron densities compared with reduced recombination.

To combine these effects with increasing the magnetic field at the target in Magnum-PSI, the position of the target was moved towards the source (beyond the TS position) so that the magnetic field at the target position increased to 0.4 from 0.3 T, while the gas flow was decreased from 17 to 10 Pa m³ s $^{-1}$ (a reduction in background pressure from 0.5 to 0.35 Pa). In this case, the heat flux increased to 128 \pm 12 MW m $^{-2}$ at a slightly lower input current of 1.25 kA, consistent with the expected scaling. As Magnum-PSI is due to be upgraded to a superconducting magnet, which will have a field at the target position of up to 2.5 T [36], this would translate to a heat flux in Magnum-PSI of \sim 550 MW m $^{-2}$ for a 1.6 kA pulse at a gas flow of 17 Pa m 3 s $^{-1}$, or approximately 950 MW m $^{-2}$

at $10 \,\mathrm{Pa}\,\mathrm{m}^3~\mathrm{s}^{-1}$. A heat flux of several $\mathrm{GW}\,\mathrm{m}^{-2}$ will be possible if the power dependence observed in section 5 for high input currents continues at higher values, but this is contingent upon the performance and survival of the source under these conditions. So far, the upper current limit for good coating lifetime is yet to be reached, which suggests higher heat fluxes should be achievable.

Upgrading to this higher and straight magnetic field will produce a more realistic replication of ITER-like ELMs in terms of heat and particle fluxes than has previously been achieved in Pilot-PSI or Magnum-PSI. Differences which will be expected to remain are that the transient heating profile could be narrower, which would lead to higher thermal shock stresses, and that the ion energies will be lower in Magnum-PSI than at the ITER divertor strike points.

Predictions from RACLETTE show that at the inboard side of the divertor outer strike point the HWHM of the temperature profile will be around 20 mm in steady state [37], while any footprint broadening expected for ELMs is not certain [8]. However, the λ_q used in [37] was 5 mm, while recent scaling law calculations predict that λ_q could be as low as 1 mm [38], which would probably reduce this HWHM value. In Magnum-PSI the HWHM will be around 5 mm, so that the thermal gradients may be somewhat higher for large spreading, but nonetheless of similar magnitude. By tilting the target the power can be spread to match the ITER divertor thermal gradients if this effect is crucial, although at the cost of reduced heat flux.

The ion energies expected in ITER are $\sim 100\,\mathrm{eV}$ during type-I ELMs, while those produced using the MERS system in this case are expected to be around 10–15 eV. To improve on this situation, a pulsed bias system will be installed, which will permit transient biasing up to $-1000\,\mathrm{V}$ in combination with the pulsed source and thus reach much higher ion energies. This should also lead to an increase in heat flux, which will contribute towards reaching the ITER-relevant ELM regime.

7. Conclusion

A multiple-pulse ELM replication system has been successfully constructed for the Magnum-PSI and Pilot-PSI linear plasma devices. This transiently delivers a greatly increased current to the source by discharging a set of coupled capacitor banks, which lead to Ohmic heating and increased ionization of the plasma. A single-pulse Thomson scattering measurement mode was implemented in Magnum-PSI to determine the transient increase in electron density and temperature at the target position of up to $1 \times 10^{21} \, \mathrm{m}^{-3}$ and $5 \, \mathrm{eV}$ respectively for a time period $\sim 0.5 - 1.5 \, \mathrm{ms}$. This system therefore combines greatly increased particle and heat fluxes to the target, which well replicates the conditions occurring during an ELM pulse in the divertor region.

Heat fluxes to a tungsten surface were determined using the THEODOR code to be up to $\sim 130\,\mathrm{MW\,m^{-2}}$ for a peak pulse current of 1250 A in the source in Magnum-PSI. The time evolution of the heat flux agreed well with that determined using the Thomson scattering measurements. From these results a value for the sheath heat transmission factor was

determined, and found to be $\gamma \approx 7.7$, which agrees well with values calculated from sheath theory. The surface heat flux was found to be strongly dependent on magnetic field strength at the target position and on the source gas flow rate, and heat fluxes of more than $600\,\mathrm{MW\,m^{-2}}$ were found when the MERS system was used in a $1.6\,\mathrm{T}$ field in Pilot-PSI under similar experimental conditions to Magnum-PSI. Extrapolation to the fields expected after installation of a high-field superconducting magnet in Magnum-PSI indicates that heat fluxes in the gigawatt per square metre region should be reached.

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References

- Janeschitz G, Borrass K, Federici G, Igitkhanov Y,
 Kukushkin A, Pacher H D, Pacher G and Sugihara M 1995
 J. Nucl. Mater. 220–222 73–88
- [2] ITER Physics Expert Group on Divertor et al 1999 Nucl. Fusion 39 2391
- [3] Loarte A et al 2007 Nucl. Fusion 47 \$203-63
- [4] Baldwin M J and Doerner R P 2010 J. Nucl. Mater. 404 165-73
- [5] Kajita S, Yoshida N, Yoshihara R, Ohno N and Yamagiwa M 2011 J. Nucl. Mater. 418 152–8
- [6] Miyamoto M, Nishijima D, Ueda Y, Doerner R, Kurishita H, Baldwin M J, Morito S, Ono K and Hanna J 2009 Nucl. Fusion 49 065035
- [7] Alimov V K, Shu W M, Roth J, Sugiyama K, Lindig S, Balden M, Isobe K and Yamanishi T 2009 Phys. Scr. T138 014048
- [8] Loarte A et al 2007 Phys. Scr. T128 222-8
- [9] Kajita S, Ohno N, Takemure S, Sakaguchi W and Nishishima D 2007 Appl. Phys. Lett. 91 261501
- [10] Umstadter K R, Doerner R and Tynan G 2009 J. Nucl. Mater. 386–388 751–5
- [11] De Temmerman G, Zielinski J J, van Diepen S, Marot L and Price M 2011 Nucl. Fusion 51 073008
- [12] Zielinski J J, van der Meiden H J, Morgan T W, Schram D C and De Temmerman G 2012 Plasma Sources Sci. Technol. 21 065003
- [13] Maecker H 1956 Z. Naturforsch. a 11 457
- [14] van der Sanden M C M, Janssen G M, de Regt J M, Schram D C and van der Mullen J A M 1992 Rev. Sci. Instrum. 63 3369
- [15] van Eck H J N, Kleyn A W, Lof A, van der Meiden H J, van Rooij G J, Scholten J and Zeijlmans van Emmichoven P A 2012 Appl. Phys. Lett. 101 224107
- [16] Scholten J et al 2013 Fusion Eng. Des. 88 1785–8
- [17] De Temmerman G et al 2013 Fusion Eng. Des. 88 483–387
- [18] Spitzer L 1956 *Physics of Fully Ionized Gases* (New York: Interscience)
- [19] van der Meiden H J et al 2012 Rev. Sci. Instrum. 83 123505
- [20] Zielinski J J, Al R, van der Meiden H, Melissen W, Rapp J and De Temmerman G 2011 J. Nucl. Mater. 415 S70–3

- [21] Herrmann A, Junker W, Gunther K, Bosch S, Kaufmann M, Neuhauser J, Pautasso G, Richter T and Schneider R 1995 Plasma Phys. Control. Fusion 37 17
- [22] Greuner H, Böswirth B, Eich T, Herrmann A, Maier H and Sieglin B 2014 Phys. Scr. T159 014003
- [23] Eich T Andrew P, Herrmann A, Fundamenski W, Loarte A, Pitts R A and JET-EFDA Contributors 2007 Plasma Phys. Control. Fusion 49 573–604
- [24] Herrmann A 2001 28th EPS Conf. on Controlled Fusion and Plasma Physics (Funchal, Portugal) P5.104
- [25] Herrmann A, Balden M, Bohmeyer W and Hildebrandt D 2004 *Phys. Scr.* **T111** 98–100
- [26] Delchambre E, Counsell G and Kirk A 2009 Plasma Phys. Control. Fusion 51 055012
- [27] 2013 ANSYS® Academic Research, Release 14.5.7
- [28] Carslaw H S and Jaeger J C 1959 Conduction of Heat in Solids 2nd edn (London: Oxford University Press)
- [29] Stangeby P C 2000 The Plasma Boundary of Magnetic Fusion Devices (Bristol: Institute of Physics)

- [30] van den Berg M, Bystrov K, Pasquet R, Zielinski J and De Temmerman G 2013 J. Nucl. Mater. 438 S431–4
- [31] Shumack A E, Veremiyenko V P, Schram D C, de Blank H J, Goedheer W J, van der Meiden H J, Vijvers W A J, Westerhout J, Lopes Cardozo N J and van Rooij G J 2008 Phys. Rev. E 78 046405
- [32] De Temmerman G, Zielinski J J, van der Meiden H, Melissen W and Rapp J *Appl. Phys. Lett.* **97** 081502
- [33] Zielinski J J 2013 A high power pulsed plasma system for material testing under simultaneous continuous and transient loads *PhD Thesis* Eindhoven University of Technology, The Netherlands (http://alexandria.tue.nl/ extra2/748465.pdf)
- [34] Ohno N, Ezumi N, Takamura S, Krasheninnikov S I and Pigarov A Y 1998 *Phys. Rev. Lett.* **81** 818–21
- [35] Yu Pigarov A 2002 Phys. Scr. **T96** 16–31
- [36] Rapp J et al 2010 Fusion Eng. Des. 85 1455-9
- [37] Pitts R et al 2013 J. Nucl. Mater. 438 S48-S56
- [38] Eich T et al 2013 Nucl. Fusion 53 093031